Mr. Mano Nazar Site Vice President Prairie Island Nuclear Generating Plant Nuclear Management Company, LLC 1717 Wakonade Drive East Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1 - UPCOMING

STEAM GENERATOR TUBE INSERVICE INSPECTION (TAC NO. MB6393)

Dear Mr. Nazar:

We would like to arrange a telephone conference with members of your staff to discuss the ongoing results of the steam generator (SG) tube inspections that will be conducted during the upcoming refueling outage (RFO-22) at Prairie Island Nuclear Generating Plant, Unit 1. This telephone conference, which is strictly voluntary on your part, should occur after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. Enclosed is a list of discussion points to facilitate this telephone conference.

Inservice inspections of SG tubes play a vital role in assuring that adequate structural integrity of the tubes is maintained. In response to a recommendation made in a report by the Nuclear Regulatory Commission's (NRC's) Indian Point 2 Lessons Learned Task Force, the NRC staff plans to document a brief summary of the telephone conference, as well as any material that you may provide in support of the call.

If you have any questions regarding this matter, please contact me at (301) 415-1392.

Sincerely,

/RA/

Tae Kim, Senior Project Manager, Section 1 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-282

Enclosure: Discussion Points

cc w/encl: See next page

November 13, 2002

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*See previous concurrence

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|--------|------------|-------------|----------|------------|
| NAME | TKim | RBouling | LLund | LRaghavan |
| DATE | 11/08/02 | 10/30/02 | 11/08/02 | 11/08/02 |

Prairie Island Nuclear Generating Plant, Units 1 and 2

CC:

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Prairie Island Indian Community
ATTN: Environmental Department
5636 Sturgeon Lake Road
Welch, MN 55089

Mr. Roy A. Anderson Executive Vice President and Chief Nuclear Officer Nuclear Management Company, LLC 700 First Street Hudson, WI 54016

Nuclear Asset Manager Xcel Energy, Inc. 414 Nicollet Mall Minneapolis, MN 55401

STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

DOCKET NO. 50-282

The following discussion points have been prepared to facilitate an upcoming telephone conference between the Nuclear Regulatory Commission (NRC) and the Nuclear Management Company, LLC (the licensee), to discuss the ongoing results of the steam generator (SG) tube inspections that will be conducted during the upcoming refueling outage (RFO-22) at the Prairie Island Nuclear Generating Plant, Unit 1. This phone call is scheduled to occur towards the end of the planned SG tube inspection interval, but before the SG inspection activities have been completed.

- 1. Discuss whether any primary to secondary leakage existed in Unit 1 prior to shutdown.
- 2. Discuss the results of secondary side pressure tests.
- For each SG, provide a general description of areas examined, including the expansion criteria utilized and the type of probe used in each area. Also, be prepared to discuss your inspection of the tube within the tubesheet, particularly the portion of the tube below the expansion/transition region.
- 4. Provide a summary of the number of indications identified to-date of each degradation mode and steam generator tube location (i.e., tube support plate, top-of-tubesheet, etc.). Also provide information, such as voltages, and estimated depths and lengths of the most significant indications.
- 5. Describe repair/plugging plans for the SG tubes that meet the repair/plugging criteria.
- 6. Discuss the previous history of SG tube inspection results, including any "look backs" performed. Specifically for significant indications or indications where look backs are used in support of dispositioning (i.e., manufacturing burnish marks).
- 7. Discuss, in general, new inspection findings (i.e., degradation mode or location of degradation new to this unit).
- 8. If SGs contain Alloy 600 thermally treated tubing, discuss actions taken (if any) based on recent findings at Seabrook.
- 9. Discuss your use or reliance on inspection probes (eddy current or ultrasonic) other than bobbin and typical rotating probes, if applicable.
- 10. Describe in-situ pressure test plans and results, if applicable and available, including tube selection criteria.

- 11. Describe tube pull plans and preliminary results, if applicable and available; include tube selection criteria.
- 12. Discuss the assessment of tube integrity for the previous operating cycle (i.e., condition monitoring).
- 13. Discuss the assessment of tube integrity for next operating cycle (i.e., operational assessment).
- 14. Provide the schedule for steam generator-related activities during the remainder of the current outage.